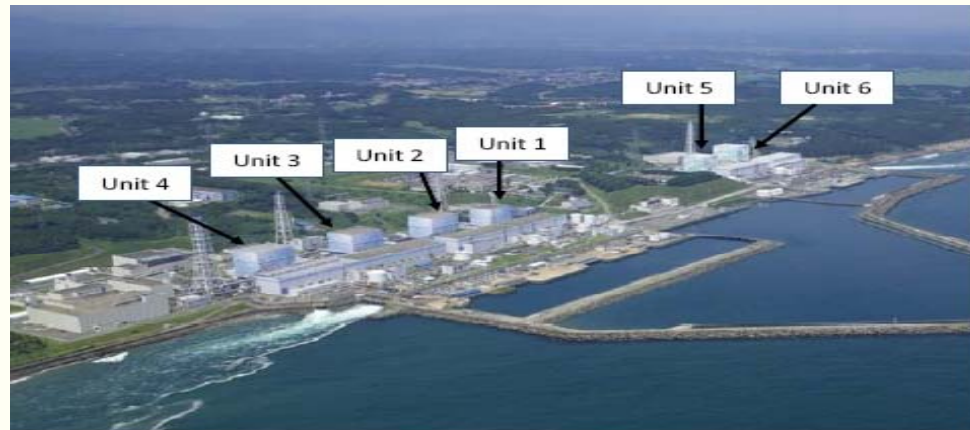


Lesson learned so far - Fukushima-I Nuclear Power Plants Accidents



**Presented by: Muhammad Ayub
Project Director**

Pakistan Nuclear Regulatory Authority

**INTERNATIONAL SEMINAR ON NUCLEAR SAFETY AND
SECURITY CHALLENGES OF THE 21ST CENTURY**

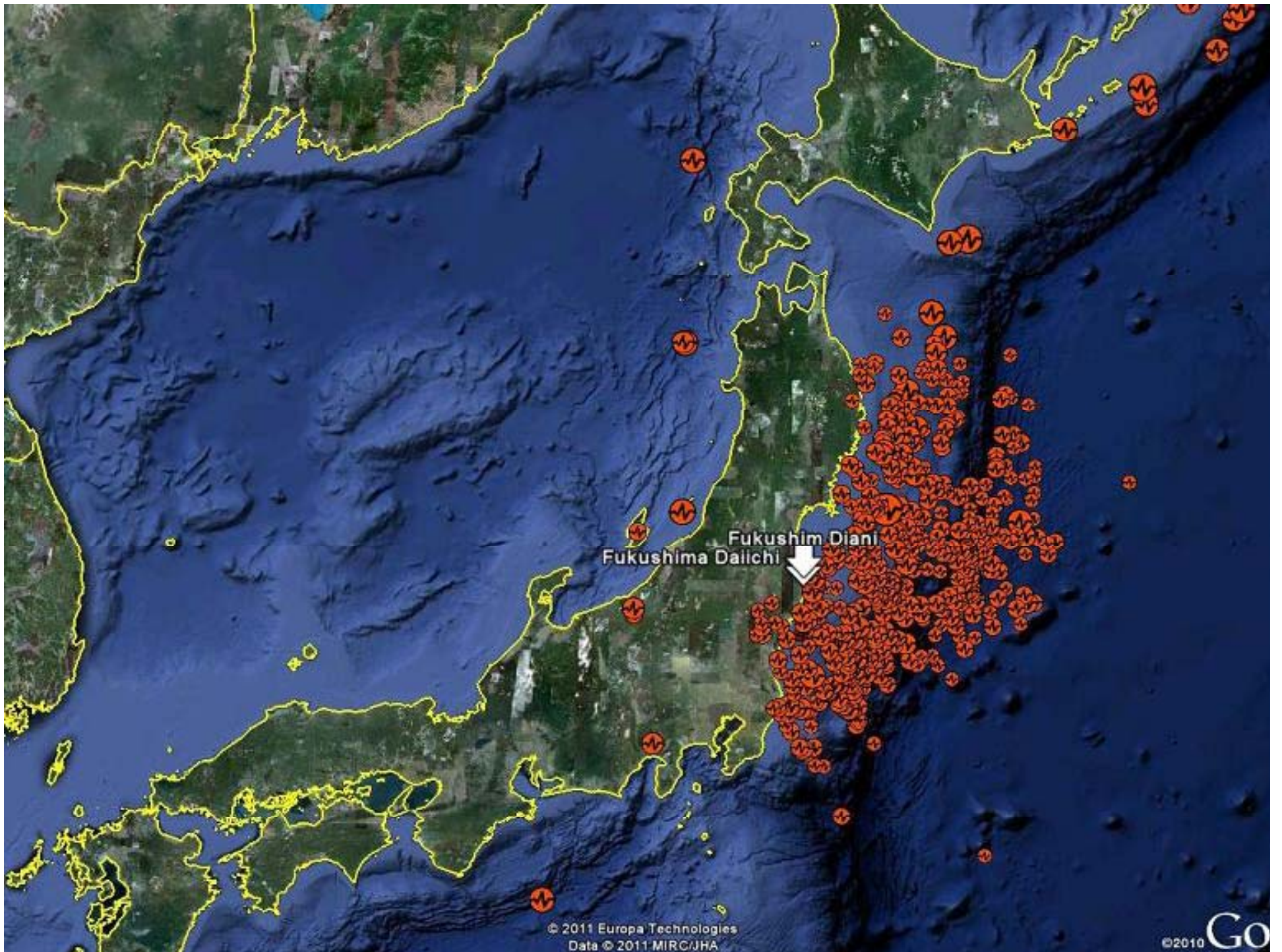
April 21 – 23, 2011





Earthquake Details

- March 11, 2011 at 05:46:23 AM UTC
- Magnitude of Earthquake - 9.0
- Predicted Earthquake – 7.5 to 8.0
- Depth of Earthquake – 32 Km
- Distance of Fukushima from epicenter – 177 Km
- Fault moved upward – 30 to 40 m and slipped over an area approximately 300 Km long and 150 Km wide





Onagawa

- Unit1: 524 MW, 1984-
- Unit2: 825 MW, 1995-
- Unit3: 825 MW, 2002-

Fukushima I

- Unit1: 460 MW, 1971-
- Unit2: 784 MW, 1974-
- Unit3: 784 MW, 1976-
- Unit4: 784 MW, 1978-
- Unit5: 784 MW, 1978-
- Unit6: 1,100 MW, 1979-

Fukushima II

- Unit1: 1,100 MW, 1982-
- Unit2: 1,100 MW, 1984-
- Unit3: 1,100 MW, 1985-
- Unit4: 1,100 MW, 1987-

Tokai II (1,100 MW, 1978-)



Tsunami Details

- Travel time - 55 min to Fukushima Unit 1
- Height of Tsunami wave
 - Fukushima –I 14m
 - Fukushima -II 12m
- Fukushima-I designed for tsunami of ~ 5.7m
- Fukushima- II designed for tsunami of ~ 5.2 m





Fukushima BWR NPP

▶ Reactor Service Floor
(Steel Construction)

▶ Concrete Reactor Building
(secondary Containment)

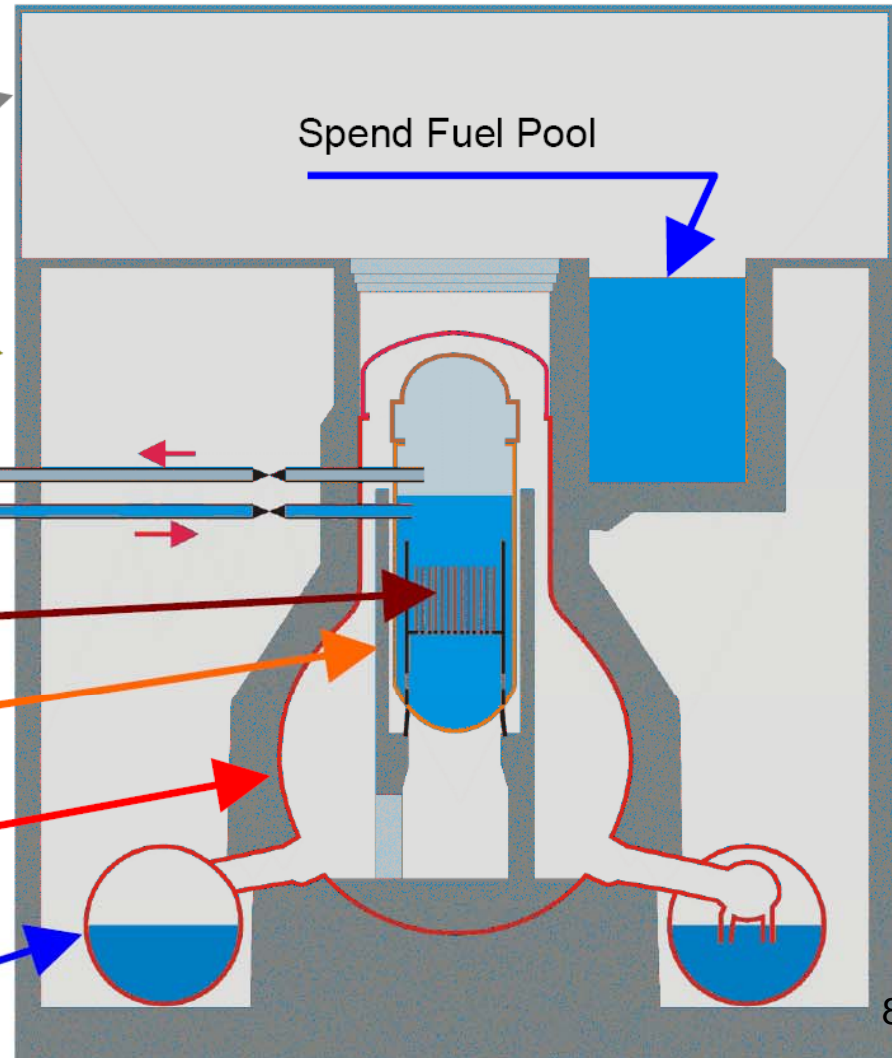
Fresh Steam line
Main Feedwater

▶ Reactor Core

▶ Reactor Pressure Vessel

▶ Containment (Dry well)

▶ Containment (Wet Well) /
Condensation Chamber



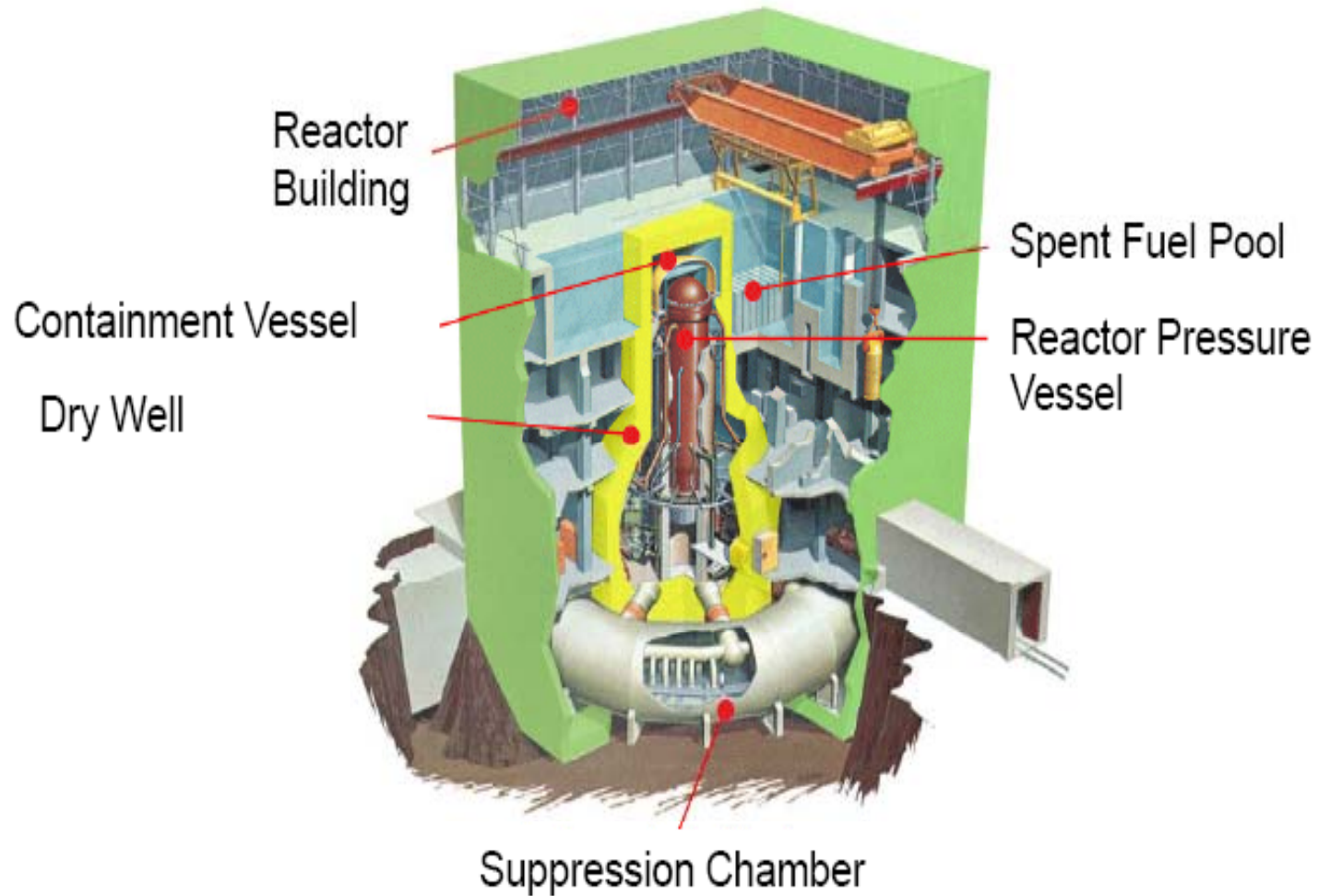


Service Floor (Reactor Building)

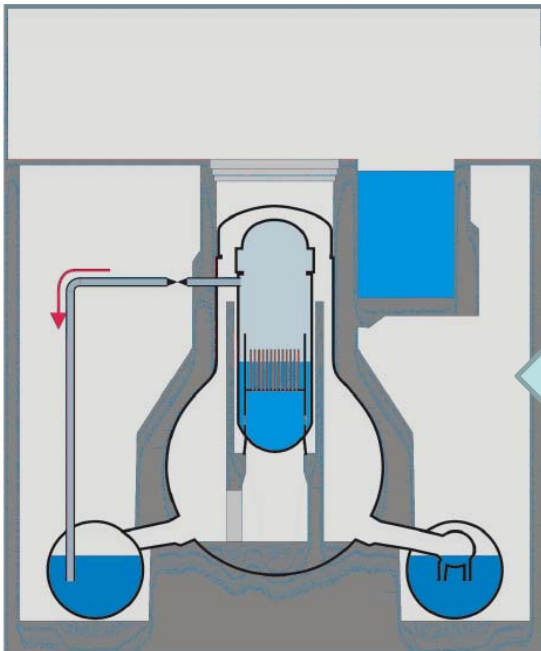
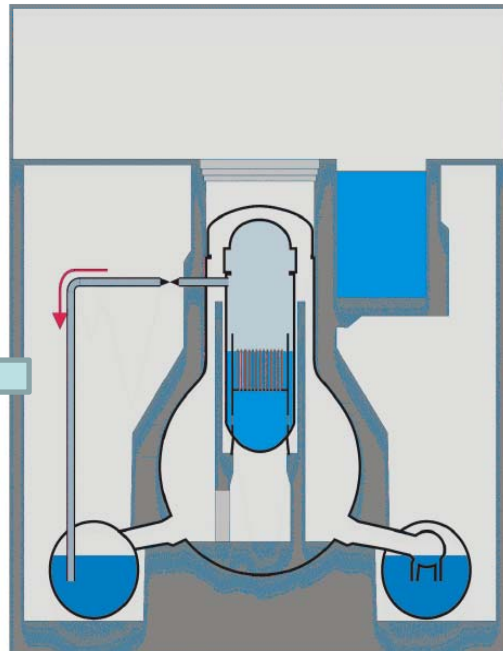
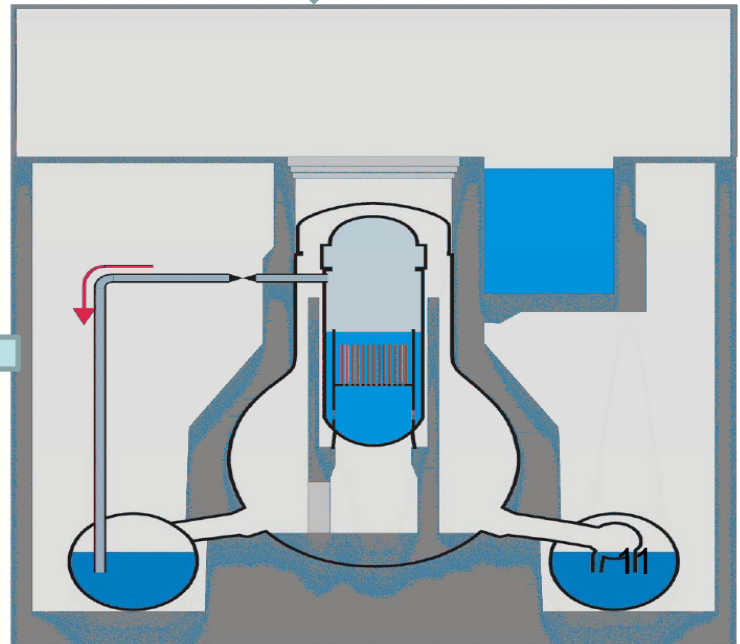
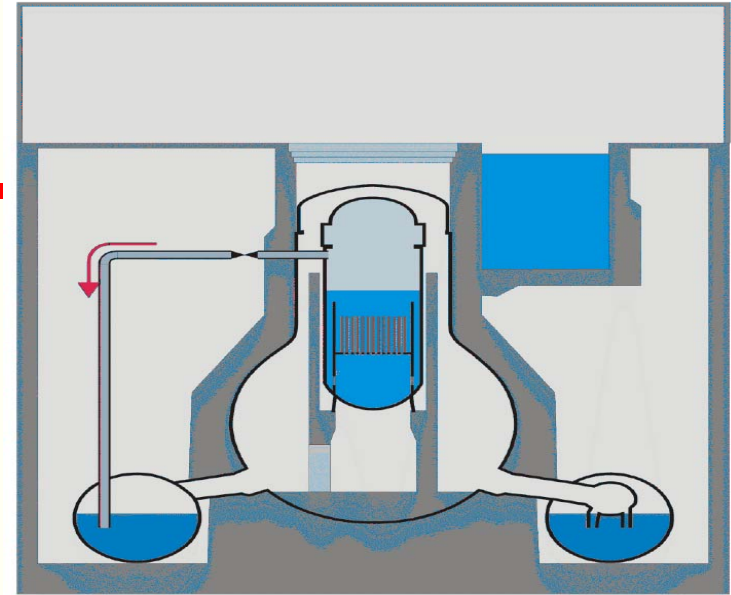




Overview of Mark-1 Type BWR (Unit 1,2,3 and 4)

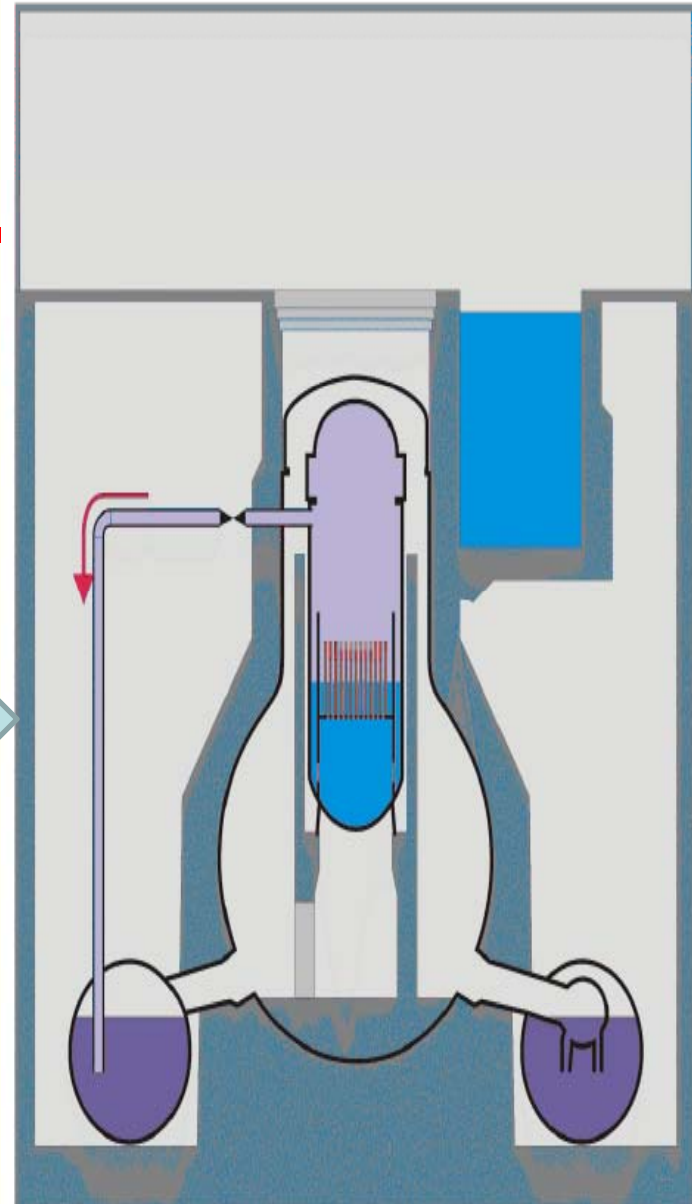


- ❖ Decay Heat produces still steam in
- ❖ Reactor pressure Vessel (RPV)-
- ❖ Pressure rising in RPV
- ❖ Opening the steam relieve valves
- ❖ Discharge Steam into the Wet-Well
- ❖ Descending of the Liquid Level in the Reactor pressure vessel



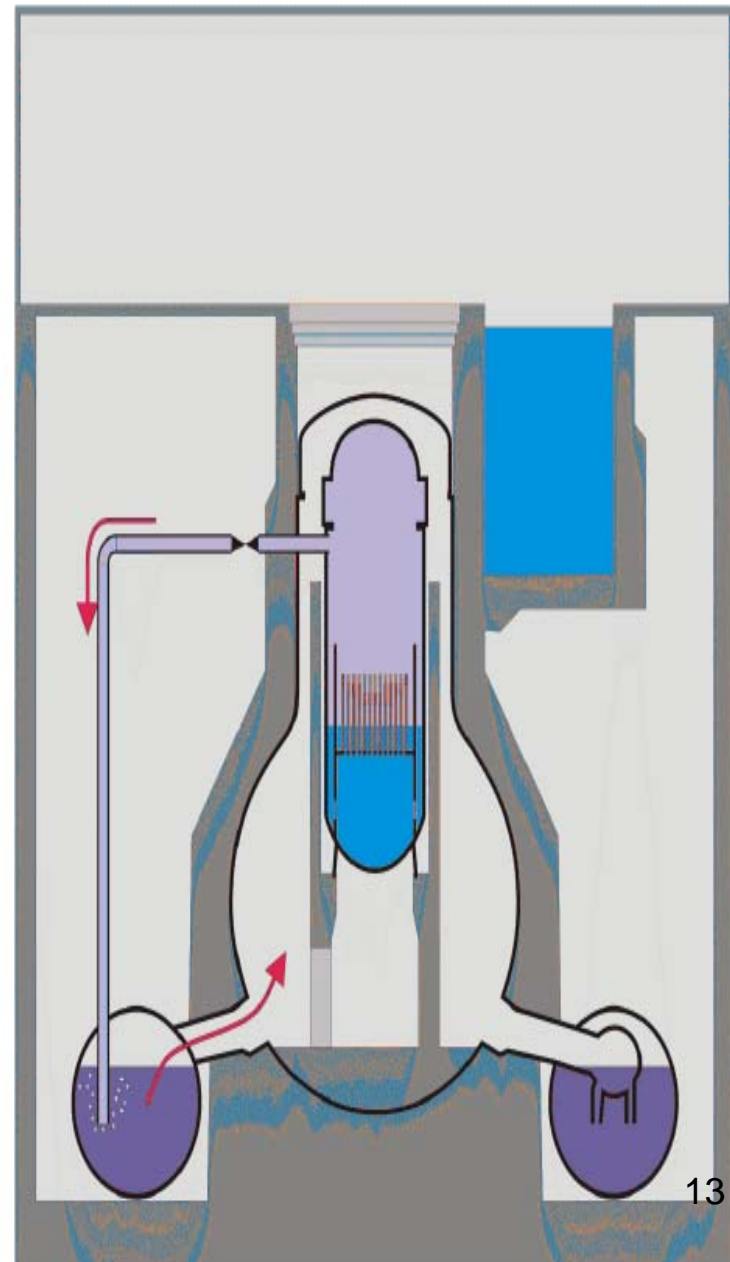


- ❖ Core exposed and Cladding temperatures rise, but still no significant core damage
- ❖ ~2/3 of the core exposed Cladding temperature exceeds ~900°C
- ❖ Ballooning / Breaking of the cladding
- ❖ Release of fission products from the fuel rod gaps





- ❖ Release of fission products Xenon, Cesium, Iodine,...during melt down. Uranium/Plutonium remain in core
- ❖ Airborne Aerosols Discharge through valves into water of the condensation chamber
- ❖ Pool scrubbing binds a fraction of Aerosols in the water
- ❖ Xenon and remaining aerosols enter the Dry-Well. Design Pressure 4-5 bar
- ❖ Actual pressure up to 8 bars
- ❖ Normal inert gas filling (Nitrogen)
- ❖ Hydrogen from core oxidation
- ❖ Boiling condensation chamber (like a pressure cooker)
- ❖ Depressurization of the containment
- ❖ Unit 1: 12.3. 4:00
- ❖ Unit 2: 13.3 00:00
- ❖ Unit 3: 13.3. 8.41



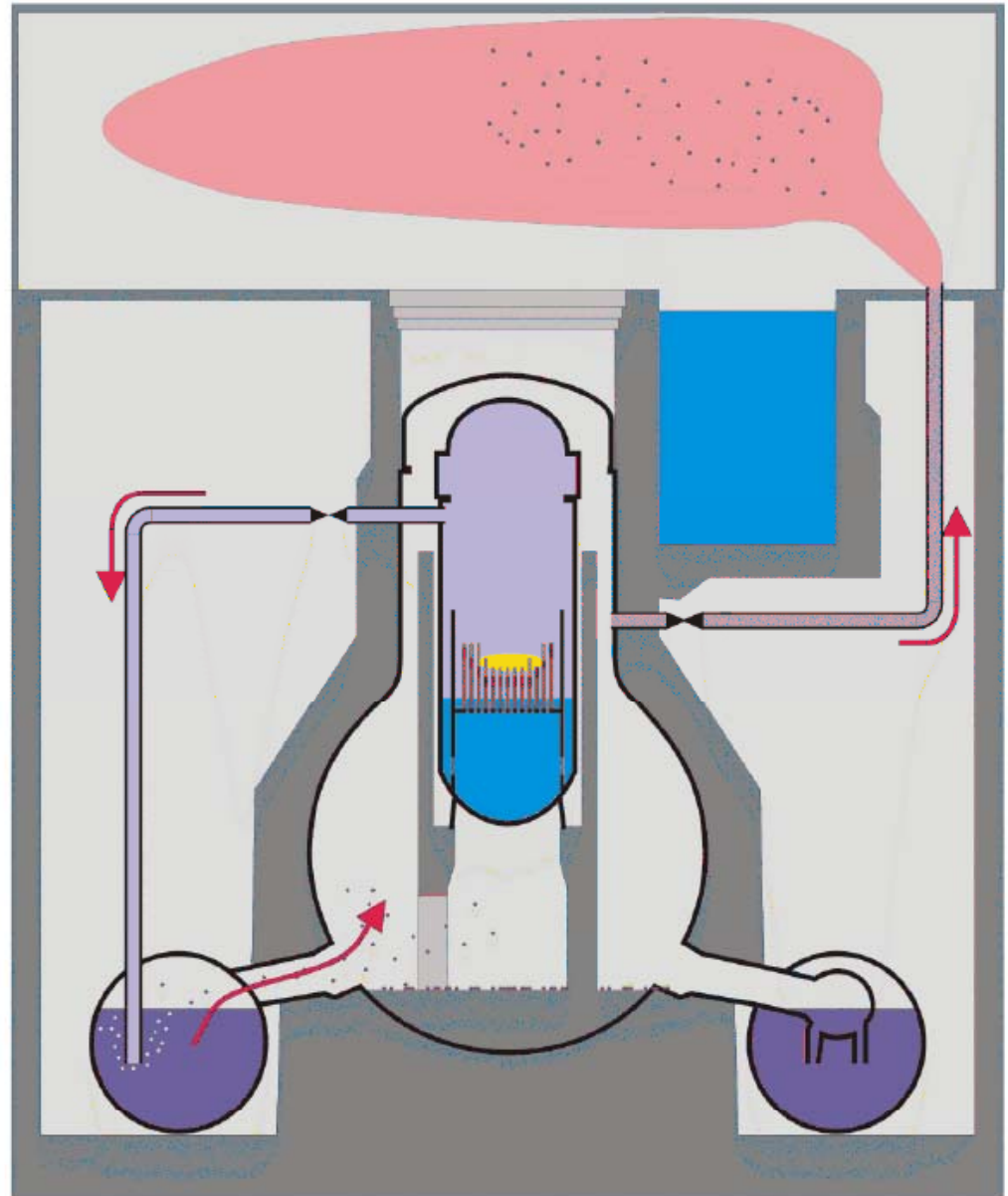


Positive and negative Aspects of depressurizing the containment

- ❑ Removes Energy from the Dry Well (only way left).
- ❑ Reducing the pressure to ~4 bar.
- ❑ Release of small amounts of Aerosols (Iodine, Cesium)
- ❑ Release of all noble gases.
- ❑ Release of Hydrogen.

Gas is released into the reactor building

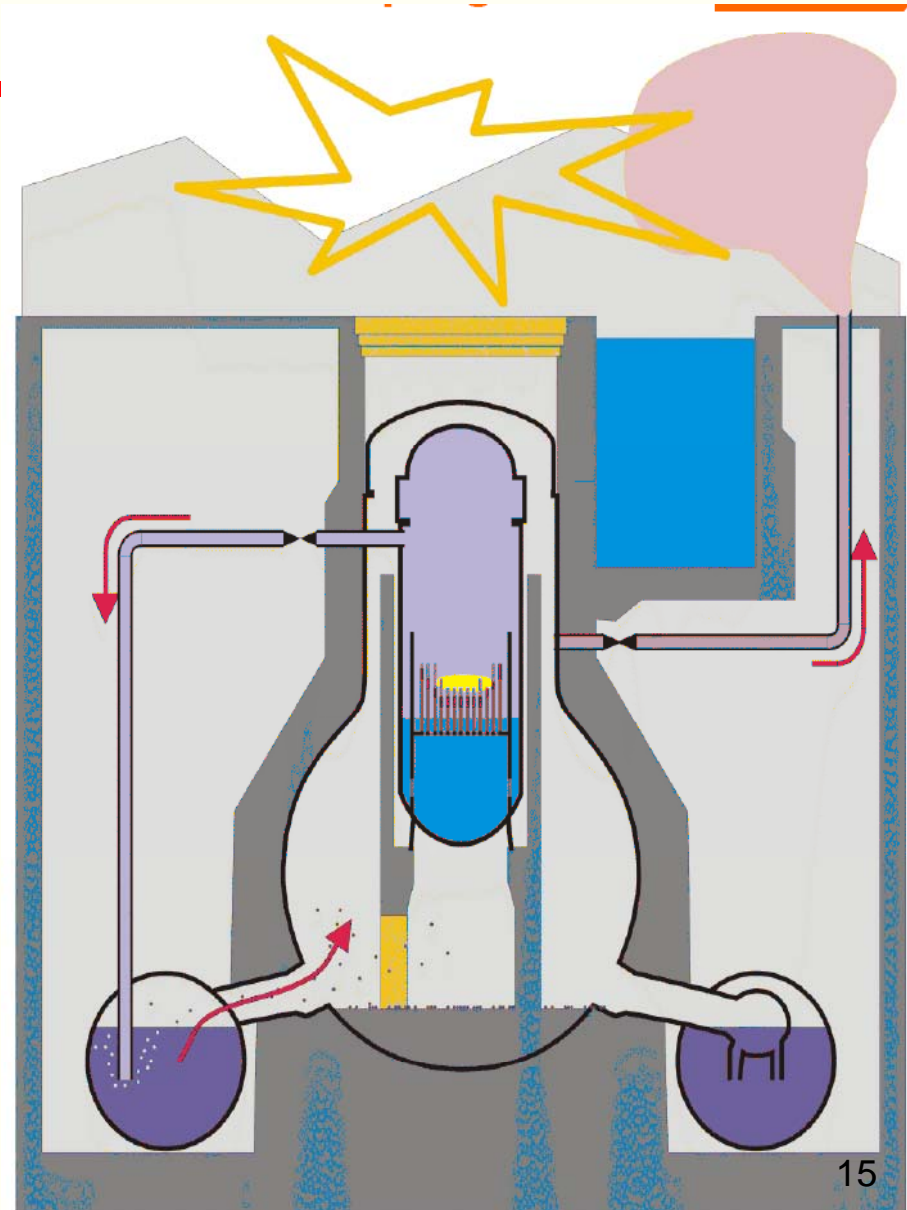
Hydrogen is flammable





Unit 1 und 3

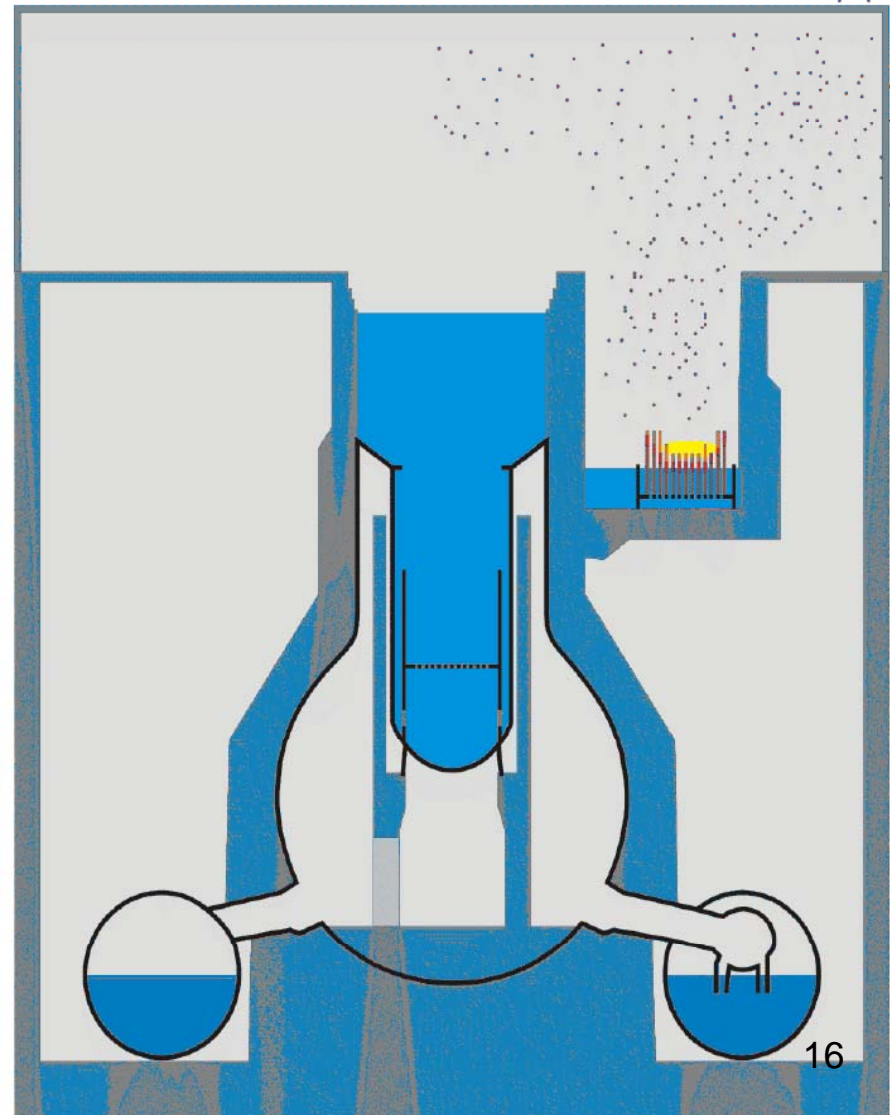
- ❖ Hydrogen burn inside the reactor building. Destruction of the steel-frame roof
- ❖ Reinforced concrete reactor building seems undamaged.





Fukushima Unit # 4

- ❖ Due to maintenance in Unit 4 entire core stored in Fuel pool
- ❖ Spend fuel stored in Pool on Reactor service floor
- ❖ Leakage of the pools due to Earthquake? Or loss of cooling. Dry-out of the pools
- ❖ Consequences - Core uncover – Oxidation of cladding – Generation of Hydrogen





General sequence of events

- Earthquake caused automatic reactor trip
- Loss of external power and Startup of Emergency Diesel Generators
- Failure of Emergency Diesel Generators within 1hr due to tsunami flooding
- Complete loss of off-site and on-site electrical power (Station Blackout) rendering all the cooling sources unavailable
- Batteries remained available for only 08 hrs
- Heat-up of core due to loss of cooling - system pressure increased



General sequence of events

- Loss of reactor core water from safety/relief valves to suppression pools
- Core uncovered and heat-up of fuel and cladding
- Metal water reaction resulting in hydrogen production
- Pressure in the containment drywell rose as wet well became very hot
- The Drywell containment was vented to reactor building
- Hydrogen accumulated at the top of reactor building
- Igniters failed as electricity was not available
- Hydrogen explosion in reactor building of units 1 & 3 occurred on 12 and 14 March respectively.
- Explosion occurred on 15 March in unit- 2, possibly damaging the pressure suppression system



General sequence of events

SEQUENCE OF EVENTS AT UNIT- 4

- The reactor core was de-fuelled in November so it was empty and all the fuel was in the spent fuel pool
- This fuel still had appreciable decay heat
- Water in the spent fuel pool boiled-off and metal water reaction started generating hydrogen gas
- Fire in spent fuel pool due to high temperature
- An explosion caused by hydrogen accumulating near the spent fuel pool damaged the rooftop area

Radioactivity released into the environment from reactor building of unit 1, 3 and 4.



Reactor No. 4

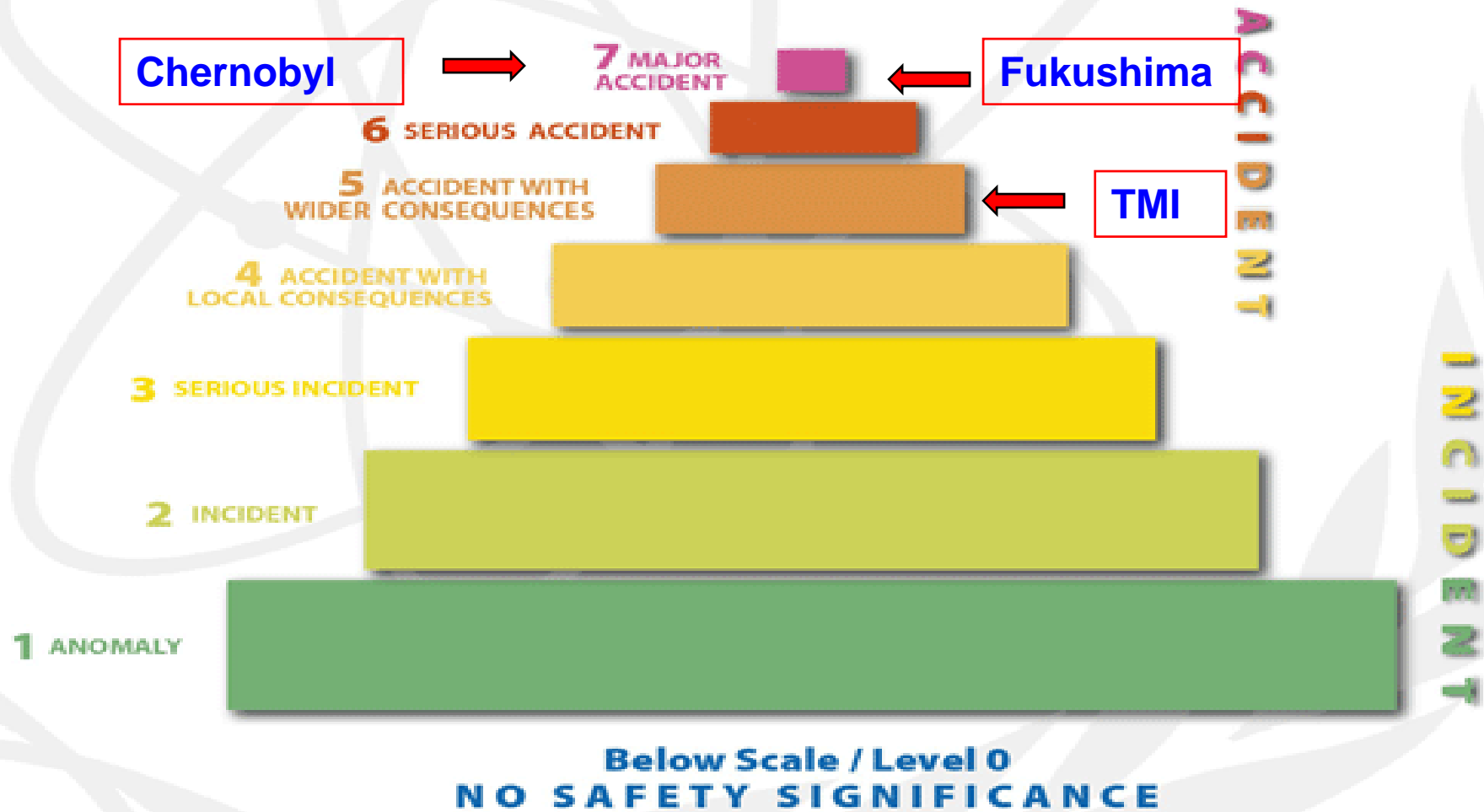
Reactor No. 3

Reactor No. 2

Reactor No. 1



International Nuclear Event Scale (INES)





MAJOR LESSONS LEARNED



Major Lessons Learned so Far

- ❖ **The details of the accident are still not available – it is too early to draw all lessons learned – major lessons listed**
- ❑ The concept of design basis for natural events needs to be revised in view of the geological and climatic changes
- ❑ Re-analyze design features of NPP against SBO
- ❑ Current regulatory requirements needs to be expanded and strengthened
- Re-evaluation of plants to withstand extreme man-made events also
- Re-evaluation of design features for controlling and removing hydrogen in the reactor and fuel buildings



MAJOR LESSONS LEARNED SO FAR

- ❑ Disaster planning should be for multiple emergencies
- ❑ Emergency plan and physical security arrangements
- ❑ Accident management strategies should be based on failure of multiple units – deal with common cause failures
- ❑ Civil liability regime should be re-assessed with regards to multiple units failure
- ❑ Layout of the plants to be revised for better handling of emergencies
- ❑ Long Term Operation (Life extension) to older Nuclear power plants- **Need to be reassessed and upgraded – before allowing further operation**